



RADIATION SHIELDING: TECHNIQUES AND LESSONS LEARNED

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N | V | 5
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- Some organizations still perform shielding calculations by hand.
- My goal is to show you some tools that will speed up this process with no loss of accuracy and with good documentation.

- NCRP Methods:
 - Report 38: Protection Against Neutron Radiation
 - Report 49: Structural Shielding Design and Evaluation for Medical Use of X Rays and Gamma Rays of Energies up to 10 MeV
 - Report 51: Radiation Protection Design Guidelines for 0.1 – 100 MeV Particle Accelerator Facilities
 - Report 144: Radiation Protection for Particle Accelerator Facilities
 - Report 147: Structural Shielding Design for Medical X-Ray Imaging Facilities
 - Report 151: Structural Shielding Design and Evaluation for Megavoltage X-and Gamma-Ray Radiotherapy Facilities
- Use Microsoft Excel or similar spreadsheet software to automate.
- Here is an example.

SPREADSHEET EXAMPLE

This is a small portion. The actual spreadsheet has over 100 lines and 8 tabs.

P: Dose goal in Uncontrolled Areas	Sv/wk	2.00E-05	100 mrem per year/50 weeks per year x 0.001 rem/mrem x 0.01 Sv/rem	NCRP 151
d _{pri}	meters	6.7056	Distance from x-ray source to 1 foot out side the primary shield wall	See drawing
W (workload) 4 MeV	Gy m ² wk ⁻¹	7973.28	4 Gy/min x 7 hrs/shift x 3 shifts/day x 7 days/wk x 60 mins/hr x 0.01 m ² x 22.6 Areas	Field size defined by Varian as 10 cm by 10 cm which equals 0.01 m ²
W (workload) 6 MeV	Gy m ² wk ⁻¹	15946.56	8 Gy/min x 8 hrs/shift x 3 shifts/day x 7 days/wk x 60 mins/hr x 0.01 m ² x 22.6 Areas	Area of a 30 degree cone at 1 meter is 0.226 m ² . Need to Increase work load by 22.6
W (workload) 9 MeV	Gy m ² wk ⁻¹	59799.6	30 Gy/min x 8 hrs/shift x 3 shifts/day x 7 days/wk x 60 mins/hr x 0.01 m ² x 22.6 Areas	Based on Varex product sheet
U: Use Factor	unitless	1	X-Ray machine points one direction only	Defined by user
T: Occupancy Factor Primary	unitless	0.025	Area is outside and is a restricted area with limited to no access	NCRP 151 Table B.1 Outdoor area
T: Occupancy Factor Adjacent Vault	unitless	0.5	Assumes area could be occupied during x-ray	NCRP 151 Table B.1 Adjacent Shielded Vault
T: Occupancy Factor - Offices and training	unitless	1	User Input	NCRP 151 Table B.1 Full Occupancy Areas
T: Occupancy Factor Corridor	unitless	0.2		NCRP 151 Table B.1 Corridor
T: Occupancy Factor Control Room	unitless	1	Assumes 24/7 operation	NCRP 151 Table B.1 Full Occupancy Areas
T: Occupancy Factor Staging Area	unitless	0.2		NCRP 151 Table B.1 Adjacent Areas

Here is the primary beam transmission factor equation used by the NCRP.

$$B_{pri} = \frac{P(d_{pri})^2}{WUT}$$

Where:

B_{pri} = transmission factor

P = Dose goal in area of interest

d_{pri} = distance to dose point

W = workload

U = Use factor for that barrier

T = Occupancy Factor in area of interest

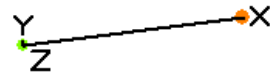
In Excel, the formula might look like this: =C4*C5^2/(C8*C9*C10)

These are all cell references. Using them automates the calculations.

- MicroShield – Available from Grove
 - Point Kernel shielding program
 - Easy to use
 - Flexible
 - Does not address neutrons
- MCNP – Only available from RSICC
 - DOS-based
 - Very detailed
 - Requires training and experience
 - Addresses photons, neutrons, electrons and other particles

MICROSHIELD EXAMPLE

MicroShield 10.06 NV5 Dade Moeller				
Date		By	Checked	
File Name		Run Date	Run Time	Duration
9MV with spectrum 1 meter.ms		July 11, 2025	6:47:46 AM	-01:59:6
Project Info				
Case Title		9MV 1m		
Description		load with spectrum		
Geometry		1 - Point		
Dose Points				
A	X	Y	Z	
#1	100.0 cm (3 ft 3.4 in)	0.0 cm (0 in)	0.0 cm (0 in)	
Shields				
Shield N	Dimension	Material	Density (g/cm ³)	
Air Gap		Air	0.00122	
Source Input: Grouping Method - User Defined Energies				
Group No.	Energy (MeV)	Activity (Photons/sec)	Point Source Photons/sec	%Energy Activity
1	1.0	3.8100e+015	3.8100e+015	20.697%
2	2.0	2.2900e+015	2.2900e+015	24.880%
3	3.0	1.0200e+015	1.0200e+015	16.623%



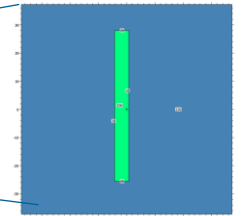
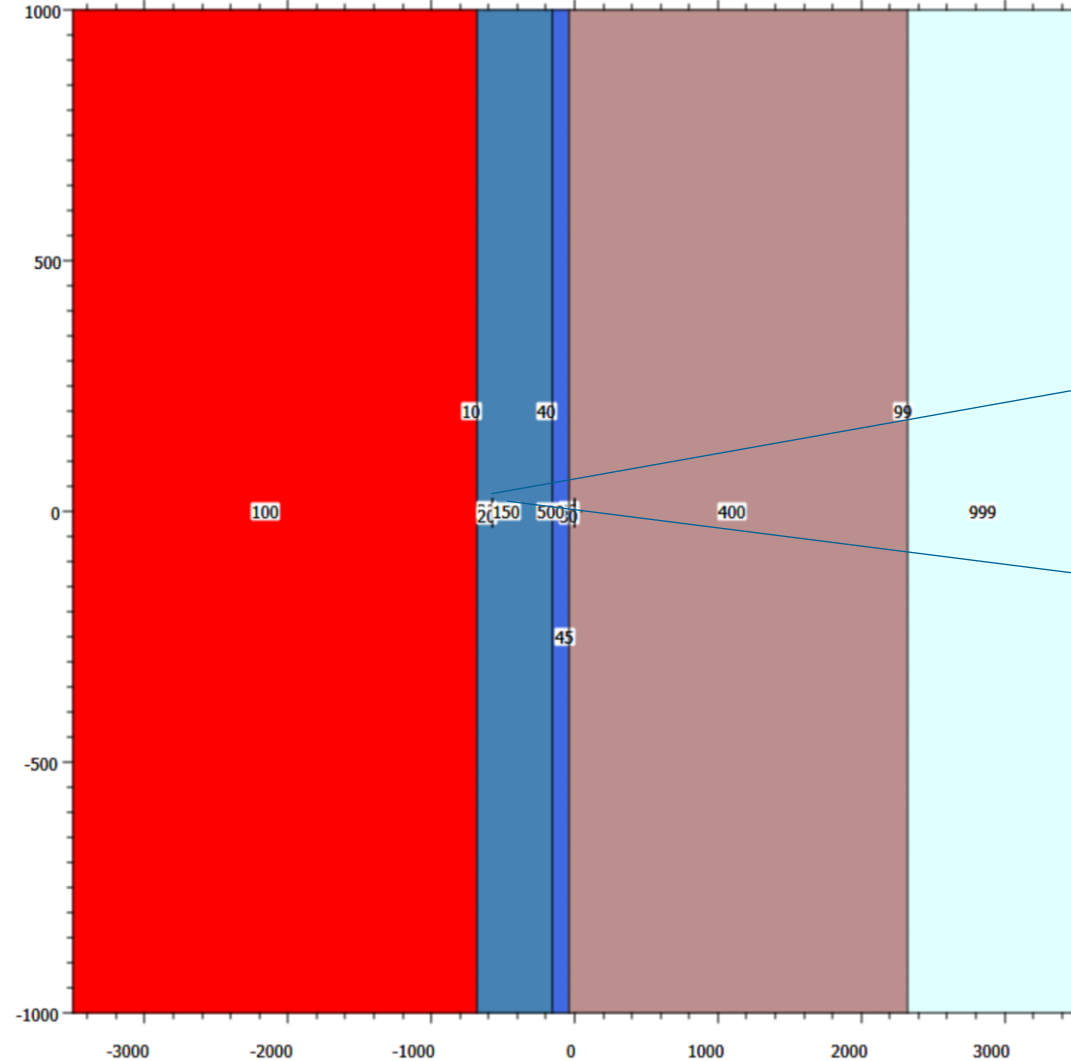
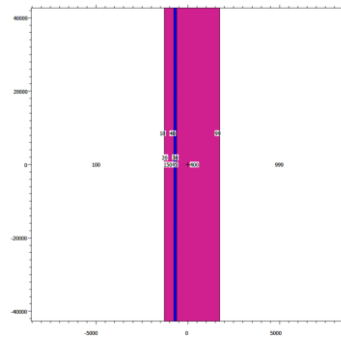
4	4.0	5.4900e+014		5.4900e+014	11.929%				
5	5.0	3.6200e+014		3.6200e+014	9.832%				
6	6.0	2.2200e+014		2.2200e+014	7.236%				
7	7.0	1.3100e+014		1.3100e+014	4.981%				
8	8.0	6.7900e+013		6.7900e+013	2.951%				
9	9.0	1.7800e+013		1.7800e+013	0.870%				
Buildup: The material reference is Air Gap. Integration Parameters									
Results									
Energy (MeV)	Activity (Photons/sec)	Fluence Rate MeV/cm ² /sec No Buildup	Fluence Rate MeV/cm ² /sec With Buildup	Exposure Rate mR/hr No Buildup	Exposure Rate mR/hr With Buildup	Absorbed Dose Rate mrad/hr No Buildup	Absorbed Dose Rate mrad/hr With Buildup	Absorbed Dose Rate mGy/hr No Buildup	Absorbed Dose Rate mGy/hr With Buildup
1.0	3.810e+15	3.009e+10	3.027e+10	5.546e+07	5.579e+07	4.841e+07	4.870e+07	4.841e+05	4.870e+05
2.0	2.290e+15	3.625e+10	3.638e+10	5.606e+07	5.625e+07	4.894e+07	4.911e+07	4.894e+05	4.911e+05
3.0	1.020e+15	2.424e+10	2.431e+10	3.289e+07	3.298e+07	2.872e+07	2.879e+07	2.872e+05	2.879e+05
4.0	5.490e+14	1.741e+10	1.745e+10	2.154e+07	2.158e+07	1.880e+07	1.884e+07	1.880e+05	1.884e+05
5.0	3.620e+14	1.436e+10	1.438e+10	1.646e+07	1.648e+07	1.437e+07	1.439e+07	1.437e+05	1.439e+05
6.0	2.220e+14	1.057e+10	1.058e+10	1.148e+07	1.149e+07	1.002e+07	1.003e+07	1.002e+05	1.003e+05
7.0	1.310e+14	7.276e+09	7.286e+09	7.573e+06	7.582e+06	6.611e+06	6.619e+06	6.611e+04	6.619e+04
8.0	6.790e+13	4.311e+09	4.316e+09	4.333e+06	4.338e+06	3.783e+06	3.787e+06	3.783e+04	3.787e+04
9.0	1.780e+13	1.272e+09	1.273e+09	1.242e+06	1.243e+06	1.084e+06	1.085e+06	1.084e+04	1.085e+04
Total	8.470e+15	1.458e+11	1.462e+11	2.070e+08	2.077e+08	1.807e+08	1.814e+08	1.807e+06	1.814e+06

MCNP INPUT FILE EXAMPLE

```
Dose Rate Example
c Geometry - cells
100 0 -10          $Void to the left of the problem
150 1000 -0.001293 10 +20 -40          $source cell
200 1000 -0.001293 -20          $detector at 1 meter
300 1000 -0.001293 -30
400 1000 -0.001293 30 45 -99
500 2000 -2.3 40 -45
999 0 99          $Void to right of the problem
```

```
c Geometry - Surfaces
10 pz -10
20 rcc 0 0 100 0 0 1 26.795
30 rcc 0 0 669.064 0 0 1 26.795
40 pz 518.16
45 pz 639.064
99 pz 3000
```

```
c sources and materials
nps 1e7
mode p e
imp:p,e 0 1 1 1 1 1 0
m1000 1001 -1.1e-3 1002 -5e-7 7014 -0.7447 7015 -2.95-3 & $Air
      8016 -0.2383 8017 -9.7e-5 18036 -3.9e-5 18038 -7.7e-6 &
      18040 -1.2383e-2
m2000 1001 -0.022094 1002 -0.000005 6000 -0.002484 8016 -0.573373 8017 -0.000232 &
      8018 -0.001326 11023 -0.015208 12000 -0.001266 13027 -0.019953 14000 -0.304627 &
      19000 -0.010045 20000 -0.42951 26000 -0.006435
sdef pos 0 0 0 par=2 erg=d1 vec 0 0 1 dir=d2 wgt=58.8235
si1 h 0 1 2 3 4 5 6 7 8 9
sp1 d 0 0.449 0.27 0.12 0.0647 0.0427 0.0262 0.0154 0.008 0.0021
SI2 -1 0.966 1.0
SP2 0 0.983 0.017
SB2 0 0 1
f11:p 20.3
f21:p 30.3
df0 iu=1 fac=-3 log ic=99
print
```



THINGS TO CONSIDER

- The NCRP method automatically assumes an x-ray spectrum.
- You must develop a spectrum for MicroShield and MCNP.
- This applies to photons for MicroShield and to all particles for MCNP.

- PNNL Compendium of Materials
 - Gives densities and constituents

- **107. Concrete, Ordinary (NIST)**

- Formula = Molecular Weight (g/mole) =
- Density (g/cm³) = 2.3 Total Atom Weight (atoms/b-cm) = 9.946E-02
- The above density is estimated to be accurate to 2 significant digits. Uncertainties are not addressed.

			Weight Fractions		Atom Fractions		Atom Densities
•	Isotopic						
•	H1	1001	-0.022094	1001	0.305287	1001	0.030365
•	H2	1002	-0.000005	1002	0.000035	1002	0.000003
•	C	6000	-0.002484	6000	0.002880	6000	0.000286
•	O16	8016	-0.573373	8016	0.499194	8016	0.049652
•	O17	8017	-0.000232	8017	0.000190	8017	0.000019

– 200-DMAMC-128170 PNNL-15870, Rev. 2

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- **Comments and References**

- Density and weight fractions from
- <http://physics.nist.gov/PhysRefData/XrayMassCoef/tab2.html>.

METHOD COMPARISON

- Beam is 9 MV
- Spectrum developed from (*Intan Dyah Ayu Permatasari et al 2019 J. Phys.: Conf. Ser. 1153 012109*)
- Shield wall is 4 feet of normal concrete, 3.37 TVL
- Dose rate is 30 Gy/m at 1 meter (manufacturer's specification)
- Dose point is at 6.7506 meters (1-foot past shield wall)

- Dose rate – unshielded

$$R = 30 \text{ Gy/m} \times 60 \text{ m/h} \times \frac{1}{(6.7506)^2} \text{ m} = 40 \text{ Gy/h} = 4,000 \text{ Rad/h}$$

- Shielded Dose Rate

$$D = D_0 \times \frac{1}{10^{TVL}} = 4000 \times \frac{1}{10^{3.37}} = 2.122 \text{ Rad/h} = 2,122 \text{ mrad/h}$$

- MicroShield
 - Point source using external source (9MV x-ray spectrum)
 - Scale the photons to get 1800 Gy/h at 1 meter
 - Dose point at 6.7506 meters
 - Source to wall distance = 5.1816 meters
 - 4-ft wall, normal concrete, 2.35 g/cm³, normal density
 - Result = 2,110 mrad/h
- MCNP
 - Same dimensions
 - Same x-ray spectrum source term
 - Result is in mrem/h per source particle
 - Calculate a scaling factor based on the mrem/h per particle at 1 meter
 - Result = 1,570 mrem/h

ADVANTAGES

NCRP Method

- Fairly inexpensive and easily available
- Routinely used and easy to understand
- Can be both performed manually and verified manually
- Straightforward conversion to Excel

DISADVANTAGES

NCRP Method

- Manpower-intensive, especially for complex or multiple points and locations
- Some methods involve graphs that do not have all energies or that need interpolation

ADVANTAGES

MicroShield

- Easy to use
- Can do multiple points in a single model
- Well-documented and accepted for photon shielding

DISADVANTAGES

MicroShield

- Cost, especially for a site license
- Does not address neutrons

ADVANTAGES

MCNP

- Handles complex geometries
- Handles all particles
- Is becoming an industry standard for calculations

DISADVANTAGES

MCNP

- Cost of program and training
- Difficult to use
- Results require interpretation
- Limited error checking
- Computer-intensive—Some runs can take hours to days to complete

- Tools are available that can automate and simplify shielding calculations.
- Need help? Resources are available.
- Tools have been used to support a wide range of efforts.
 - State agencies such as the Commonwealth of Massachusetts
 - Industrial users: Washington, West Virginia, Maryland, California
 - Medical users: Washington, California, Virginia
 - Isotope manufacturers and nuclear pharmacies: Indiana

QUESTIONS?

Contact me for more information.

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